

Nuclear I&C Systems Safety

The Principles of Nuclear Safety for Instrumentation and Control Systems Faculty of Transportation Engineering and Vehicle Engineering
Department of Control for Transportation and Vehicle Systems

Legal and Regulatory Framework

Legal framework, regulatory bodies and main standards of Nuclear Power Plants



Legal Framework

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- Act CXVI of 1996 on Atomic Energy (Atomic Act)
- Govt. Decree 118/2011. (VII. 11.) on the nuclear safety requirements of nuclear facilities and on related regulatory activities (Nuclear Safety Code)
 - Volume 1. Nuclear safety authority procedures of nuclear facilities
 - Volume 2. Management systems of nuclear facilities
 - Volume 3. Design requirements of nuclear power plants
 - Volume 3a. Design requirements of nuclear power plants (new installation)
 - Volume 4. Operation of nuclear power plants
 - Volume 5. Design and operation of research reactors
 - Volume 6. Interim storage of spent nuclear fuel
 - Volume 7. Site survey and assessment of nuclear facilities
 - Volume 8. Decommissioning of nuclear facilities
 - Volume 9. Requirements for the construction of a new nuclear installation
 - Volume 10. Nuclear Safety Code definitions
- Govt. Decree 190/2011. (IX. 19.) on physical protection requirements for various applications of atomic energy, and on the corresponding system of licensing, reporting and inspection



Regulatory Body (Licensor)

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Hungarian Atomic Energy Authority

• Responsible for the regulatory tasks in connection with



Országos Atomenergia Hivatal

- the use of atomic energy exclusively for peaceful purposes,
- the safety of nuclear facilities and transport containers,
- the security of nuclear and other radioactive materials and associated facilities.
- With the consideration of the relevant legal requirements, authorizes the licensee to perform activities in connection with the use of atomic energy.
- Regularly reviews and assesses the operation of the licensees, and the safety and security performance of the facilities. If observes any non-compliance, then it takes or order measures to its elimination.



International Guidance and Coordination

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International Atomic Energy Agency

• The IAEA is the "Atoms for Peace" organization within the United Nations family.



- Set up in 1957 as the world's centre for cooperation in the nuclear field, the Agency works with its Member States and multiple partners worldwide to promote the safe, secure and peaceful use of nuclear technologies.
- Main Work Areas
 - Nuclear Technology & Applications
 - to help countries use nuclear and isotopic techniques to promote sustainable development objectives.
 - Nuclear Safety & Security
 - to provide a strong, sustainable and visible global nuclear safety and security framework, protecting people and the environment from the harmful effects of ionizing radiation.
 - Safeguards & Verification
 - to fulfil the duties and responsibilities of the IAEA as the world's nuclear inspectorate.



IAEA Main I&C Related Standards

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Requirements

Safety Guide

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Deprecated

New

IAEA Safety Standards Series NS-R-1 (2000), Safety of Nuclear Power Plants: Design

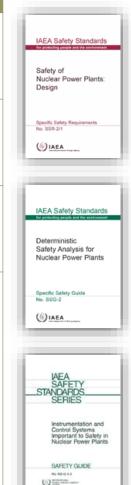
IAEA Safety Standards Series NS-R-2 (2000), Safety of Nuclear Power Plants: Operation

IAEA Safety Standards Series NS-G-1.1 (2000), Software for Computer Based Systems Important to Safety in Nuclear Power Plants

IAEA Safety Standards Series NS-G-1.3 (2002), Instrumentation and Control Systems Important to Safety in Nuclear Power Plants IAEA Safety Standards Series SSR-2/1 (2012), Safety of Nuclear Power Plants: Design Specific Safety Requirements IAEA Safety Standards Series SSR-2/2 (2011), Safety of Nuclear Power Plants: Commissioning and Operation IAEA Safety Standards Series SSG-2

(2010), Deterministic Safety Analysis for Nuclear Power Plants

IAEA Safety Standards Series SSG-39 (2016), Design of Instrumentation and Control Systems for Nuclear Power Plants (supersedes NS-G-1.1 and NS-G-1.3)



Other IAEA I&C Related Guides

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IAEA Safety Standards Series SSG-30 (2014), Safety Classification of

Structures, Systems And Components in Nuclear Power Plants

IAEA Nuclear Energy Series NP-T-3.12 (2011), Core Knowledge on

Instrumentation and Control Systems in Nuclear Power Plants

IAEA Nuclear Security Series NSS-17 (2011), Computer Security at Nuclear Facilities

IAEA Nuclear Energy Series NP-T-1.5 (2009), Protecting Against Common Cause Failures in Digital I&C Systems of Nuclear Power Plants

IAEA Nuclear Energy Series NP-T-1.4 (2009), Implementing Digital Instrumentation and Control Systems in the Modernization of Nuclear Power Plants

IAEA TECDOC-1389 (2004), Managing modernization of nuclear power plant instrumentation and control systems

IAEA TECDOC-1327 (2002), Harmonization of the licensing process for digital instrumentation and control systems in nuclear power plants







Nuclear Standards: Differences from IEC 61508

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- Mixed deterministic/probabilistic approach
 - Safety functions are classified into categories according to their impact on plant safety
 - Systems are classified into categories according to the safety functions they provide
 - Requirements are assigned to categories
 - Requirements are drawn from the plant safety design base
- Many requirements are explicitly deterministic
 - Design for reliability
 - Single failure criterion \rightarrow Redundancy
 - Common cause failure criterion \rightarrow Independence \rightarrow Diversity
 - Lack of backlash from lower category equipment



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Nuclear I&C Safety Principles

Principles, Terms and Concepts of Safety in **Nuclear Instrumentation and Control Systems**



Safety Classification of I&C Functions

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- The safety classification is usually performed using a combination of deterministic methods, probabilistic methods and engineering judgment taking into consideration:
 - The safety function(s) to be performed (to take action in response to some plant event, or to not fail in a way that would cause a hazardous event);
 - The probability of, and the safety consequences that could result from, a failure of the function;
 - The probability that the function will be needed to provide safety.
 - If the function is needed:
 - how quickly the function must respond and for how long the function must be performed;
 - the timeliness and dependability of alternative actions.
- Once I&C functions are classified, systems and components are assigned to classes according to the highest level function that they must perform.

Comparison of Different Classification Systems

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Nat. or intl. standard	Classification of the importance to safety					
IAEA NS-R-1	Systems Important to Safety			Systems Not		
IAEA INS-K-1	Safe	ty		Safet	y Related	Important to Safety
IEC 61226		System	ns Import	ant to Saf	ety	
Functions Systems	Category A Class 1		gory B ass 2	Category C Class 3		Unclassified
Canada	Category	1	L Category 2 Category 3		Category 4	
France N4	1E		2E	SH	Important to Safety	Systems Not Important to Safety
EUR	F1A (Aut.)	F1B (A./M.)		F2	Unclassified
Russian Fed.	Class 2		Class 3		Class 4 (N/I. to Safety)	
USA and		Systems Important to Safety			Non nuclear Safety	
IEEE	SR / Class 1E	(No name assigned)			Non-nuclear Safety	
R. of Korea	10	C-1 IC-2			IC-3	

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- 1) Specification of performance requirements for I&C actions is necessary to ensure that these functions are achieved over the full range of measured variables to be accommodated, with the characteristics (e.g., accuracy, response time) to produce the necessary output signal.
- 2) Design for reliability of I&C systems important to safety is necessary to prevent undue challenges to the integrity of the plant physical barriers provided to limit the release of radiation and to ensure the reliability of engineered protective systems.
 - a. Compliance with the single failure criterion
 - b. Redundancy
 - c. Diversity
 - d. Independence
- 3) Consideration of equipment failure modes (fail safe principle) is given in the design of I&C systems to make their functions more tolerant of expected failures of systems or components. The design of systems and equipment should strive to ensure that the range of possible failure modes is predictable and that the most likely failures will always place the system in a safe state.

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- 4) Control of access to I&C equipment important to safety must be established to prevent unauthorized operation or changes and to reduce the possibility of errors caused by authorized personnel.
- 5) Set point analysis is performed to ensure that I&C functions that must actuate to ensure safety do so, before the related process parameter exceeds its safe value (safety limit).
 - An analysis is necessary to calculate the point at which the I&C system must act to accomplish this. The difference between the safety limit and the set point must account for errors and uncertainties that cause a difference between the measured value acted upon by the I&C system and the actual value of the physical process.
- 6) Design for optimal operator performance is the practice of applying human factors engineering to minimize the potential for operator errors and limit the effects of such errors.
 - Human factors engineering is applied to ensure that operators have the information an controls needed for safe operation and to provide an operator friendly interface for operation, maintenance, and inspection of systems important to safety.



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- 7) Equipment qualification is a process for ensuring that the systems and equipment important to safety are capable of performing their safety functions. This process involves the demonstration of the necessary functionality under all service conditions associated with all plant design states.
- 8) Quality in the design and manufacturing of systems and equipment important to safety is necessary to demonstrate that they will perform their assigned safety functions.
- 9) Design for electromagnetic compatibility is necessary to ensure that installed systems and equipment will withstand the electromagnetic environment in a nuclear power plant.
 - This involves making appropriate provisions for the grounding, shielding and decoupling of interference.
 - The qualification of equipment for operation in the electromagnetic environment is important and is a part of equipment qualification.



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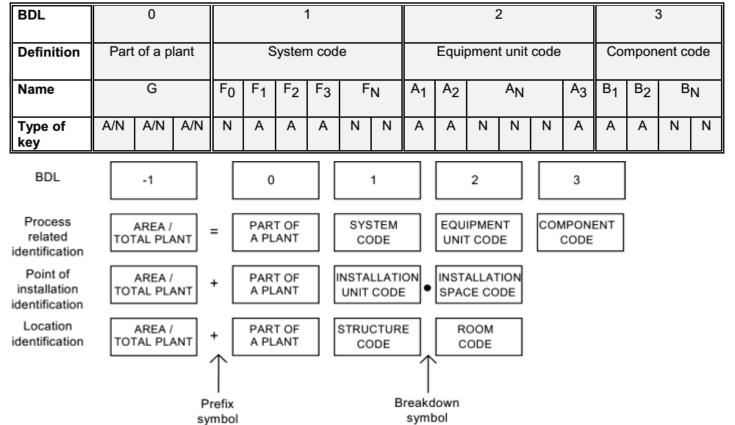
- **10)** Testing and testability provide assurance that I&C systems and equipment important to safety remain operable and capable of performing their safety tasks.
 - This principle includes both the need to provide a design that facilitates testing, calibration, and maintenance, and the establishment of programs to appropriately schedule, conduct, and learn from these activities.
- 11) Maintainability is the principle of designing I&C systems and equipment important to safety to facilitate timely replacement, repair, and adjustment of malfunctioning equipment.
 - A consequence of design for testability and maintainability is the provision of additional redundancy so that the single failure criterion continues to be met while one redundancy is removed for maintenance or testing.
- 12) Documentation of I&C functions, systems, and equipment is necessary to ensure that the plant operating organization has adequate information to ensure safe operation and maintenance of the plant and to safely implement subsequent plant modifications.
- 13) Identification of I&C functions, systems, and equipment important to safety is required to ensure that these items are properly treated during the design, construction, maintenance and operation of the plant.
 - Both the physical items, and documentation of these items should unambiguously identify their safety significance.

KKS (Kraftwerk Kennzeichnen System)

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The KKS code consists of alpha letters (A) and numbers (N). The code is divided in 4 (0-3) BDL's in the process related code and in 3 (0-2) BDL's in the point of installation code and the location code.



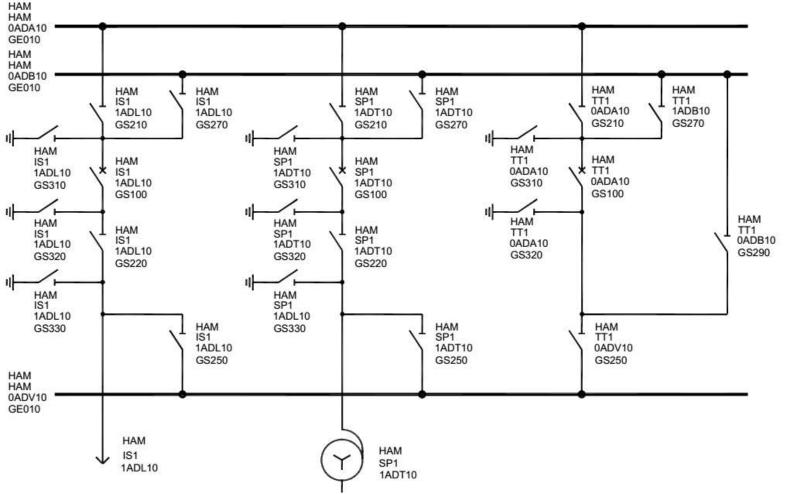


KKS Coding Example

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Example of coding line- and transformer bays, two busbars and one spare

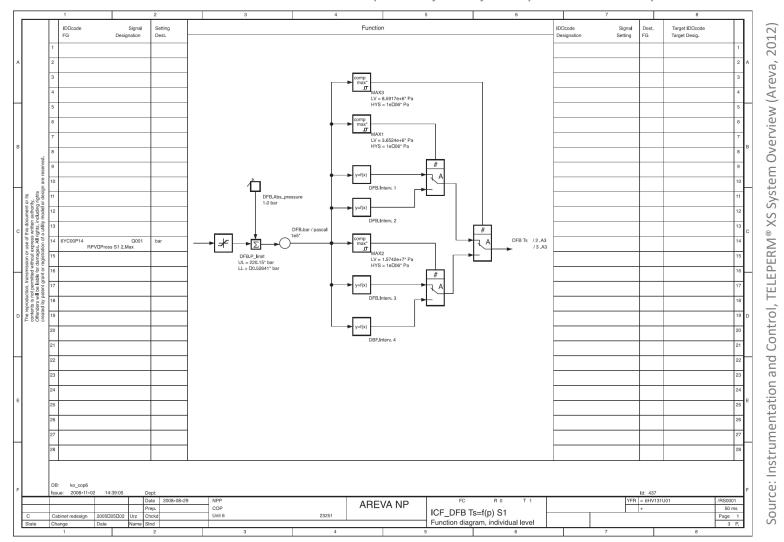


I&C System Functional Description

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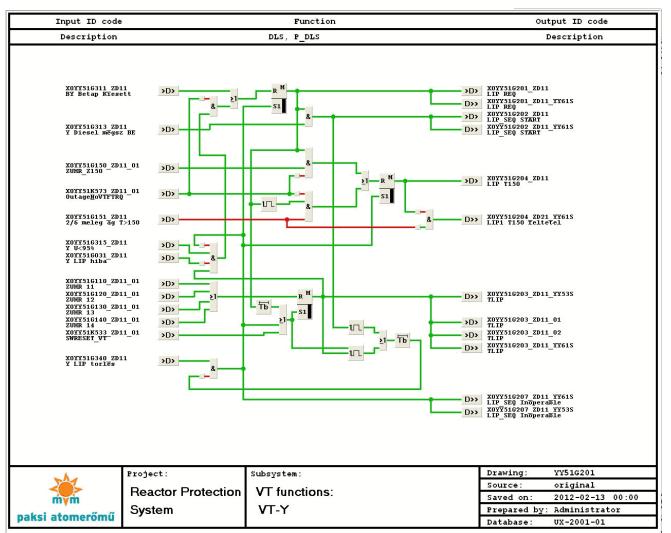


I&C Functional Specification in the Paks NPP

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Nuclear I&C Systems Safety



Defence in depth

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Definition and Comments	Relationships	Examples
A hierarchical deployment of	Provides	
different levels of diverse equipment and procedures to prevent the escalation of anticipated operational occurrences and to maintain the effectiveness of physical barriers placed between a radiation	The primary means of preventing accidents in a nuclear power plant and mitigating the consequences of accidents if they do occur is the application of the concept of defence in depth	
source or radioactive material	Requires	I&C Systems
and workers, members of the	• 5 levels	Control Systems
public or the environment, in	• 3 layers	Limitation Systems
operational states and, for some barriers, in accident conditions.	 active, passive and inherent 	 Protection Systems
	safety features	• ESFAS



Current Recent Concept of Defence-in-Depth in NPPs

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Levels of defence in depth	Objective	Essential means	Associated plant condition categories
Level 1	Prevention of abnormal operation and failures	Conservative design and high quality in construction and operation	Normal operation
Level 2	Control of abnormal operation and detection of failures	Control, limiting and protection systems and other surveillance features	Anticipated operational occurrences
Level 3	Control of accident within the design basis	Engineered safety features and accident procedures	Design basis accidents (postulated single initiating events)
Level 4	Control of severe plant conditions, including prevention of accident progression and mitigation of the consequences of severe accidents	Complementary measures and accident management	Multiple failures Severe accidents
Level 5	Mitigation of radiological consequences of significant releases of radioactive material	Off-site emergency response	



Design for reliability of I&C systems important to safety

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Necessary to prevent undue challenges to the integrity of the plant physical barriers, and to ensure the reliability of engineered protective systems.

- Compliance with the single failure criterion is a deterministic approach to ensuring that I&C systems can tolerate a random failure of any individual component, taking into account both the direct consequences of such a failure and any failures caused by events for which the system must function.
- Redundancy is the provision of multiple means of achieving a given function. It is commonly used in I&C systems important to safety to achieve system reliability goals and/or conformity with the single failure criterion.
 - For redundancy to be fully effective the redundant systems must be independent of each other.



Design for reliability of I&C systems important to safety

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- Independence prevents propagation of failures from system to system, between redundant elements within systems, and caused by common internal plant hazards.
 - Independence can be achieved through physical separation, isolation, remote location, etc.
- Diversity in I&C systems is the principle of monitoring different parameters, using different technologies, different logic or algorithms, or different means of actuation in order to provide several ways of achieving an I&C function. Diversity provides defence against common cause failures (CCF).
 - It is complementary to the plant design principle of defence in depth.
- Consideration of equipment failure modes (fail safe principle) is given in the design of I&C systems to make their functions more tolerant of expected failures of systems or components.
 - The design should ensure that the range of possible failure modes is predictable, and that the most likely failures will always place the system in a safe state.

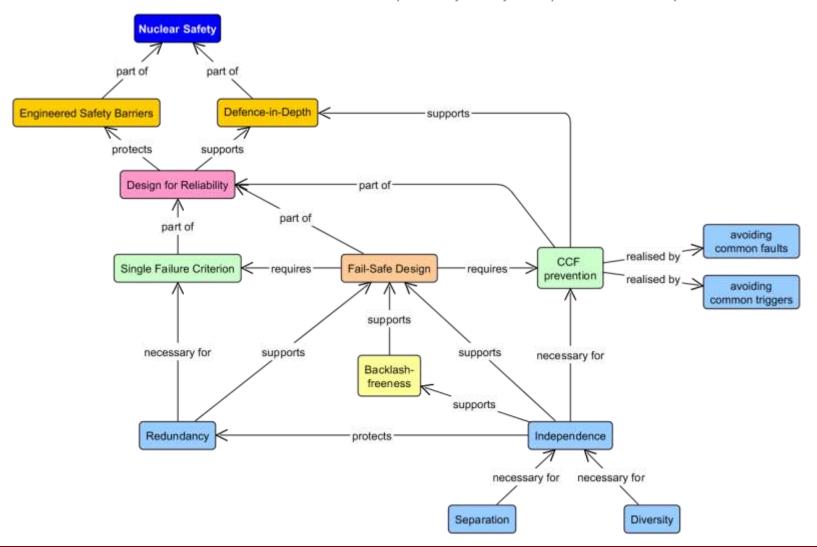


Design for Reliability Principles

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Design for reliability

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Definition and Comments	Relationships	Examples
All structures, systems and	Design features	Graded approach
components that are items important to safety be designed such that their quality and reliability are commensurate with their	 Tolerance of random failure Tolerance of common cause failures Fail-safe design 	Safety measures are applied proportional to the potential consequences of a failure.
classification.	 Independence of equipment and systems Selection of high quality equipment Testability and maintainability 	Availability Reliability Safety Confidentiality Integrity Maintainability and Threats Errors
A suitable combination of probabilistic and deterministic design criteria	Requires Safety objective 	Security Failures Fault Prevention Means Fault Tolerand Fault Remova Fault Forecas
should typically be applied.	Safety principlesRequirements and measures	

<u> SUE</u>

Fail-safe design

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Definition and Comments	Relationships	Conformance
The concept of fail-safe design	I&C systems	Verification and validation
shall be incorporated, as appropriate, into the design of systems and components important to safety.	I&C systems for items important to safety shall be designed for high functional reliability and periodic testability commensurate with the	Formal methodsDeterministic safety assessment
	safety function(s).	 Testing
Systems and components	Requires	Evidence
important to safety shall be designed for fail-safe behaviour, as appropriate, so that their failure or the failure	Single failure toleranceCommon cause failure avoidance	 Safety case
of a support feature does not	Redundancy	
prevent the performance of	 Independence 	
the intended safety function.	Diversity	



Common cause failure

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cause.

Definition and Comments	Relationships	Causes
Failure of two or more structures,		Origin
systems and components due to a single specific event or cause.		Human error(Common) dependence
For example, a design deficiency, a manufacturing deficiency, operation and		Environmental
maintenance errors, a natural	Means	Constituents
phenomenon, a human induced event, saturation of signals, or an unintended cascading effect.	IndependenceDiversity	 (Common) fault/error (Common) trigger
Common mode failure	Supported by	
Failure of two or more structures, systems and components in the same manner or mode due to a single event or	 Deterministic safety assessment Formal methods 	



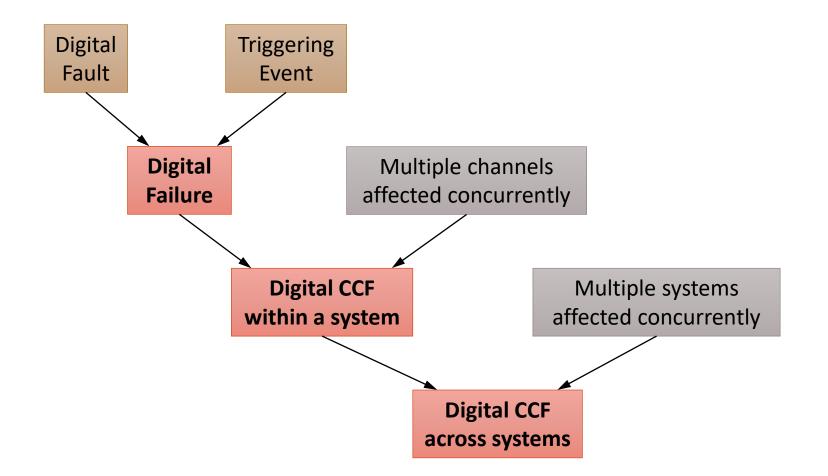
Conditions required to create a digital CCF

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Independence

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Definition and Comments	Relationships		
Safety systems should be	Provides	Examples	
 independent of safety related and non-safety systems. Independence should be provided between redundant parts of safety systems and safety-related systems. Appropriate independence should be provided between diverse functions. 	 Prevents: (1) propagation of failures from system to system or (2) propagation of failures between redundant parts within systems, and (3) common cause failures due to common internal plant hazards. 	 Separate locations (rooms) Independent cabling (paths) Analogue / Digital technology 	
Interference between safety	Means		
systems or between redundant elements of a system shall be prevented by appropriate means.	 Physical separation Electrical isolation Functional independence Independence of communication (data transfer) 		



Diversity

Definition and Comments	Relationships		
The presence of two or more redundant systems or components to perform an identified function, where the different systems or components have different attributes so as to reduce the possibility of common cause failure, including common mode failure.	Types• Human diversity• Design diversity• Software diversity• Functional diversity• Signal diversity• Equipment diversity• System diversity	 Diversity When are two systems diverse enough? 	
Examples: different operating conditions, different working principles or different design teams, different sizes of equipment, different manufacturers, types of equipment that use different physical methods.	Requires • Independence	 Examples Heterogeneity N-version programming Recovery Blocks 	



Single failure criterion

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Definition and Comments	Relationships	
A criterion (or requirement) applied	Provides	Applies to
to a system such that it must be capable of performing its task in the presence of any single failure.	Assessment is often aimed at quantifying performance measures for comparison with criteria.	Systems important to safety
The double contingency principle is, for example, such that the design for a process must include sufficient safety factors that an accident would not be possible unless at least two	Requires	
	Redundancy	
	 Independence 	
	Supported by	
unlikely and independent changes in process conditions were to occur concurrently.	 Deterministic safety assessment 	



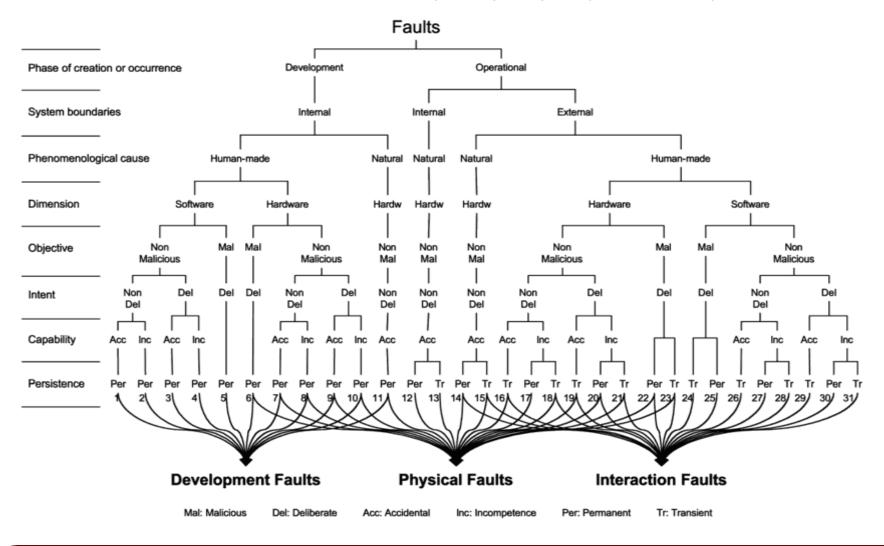
Classification of Faults

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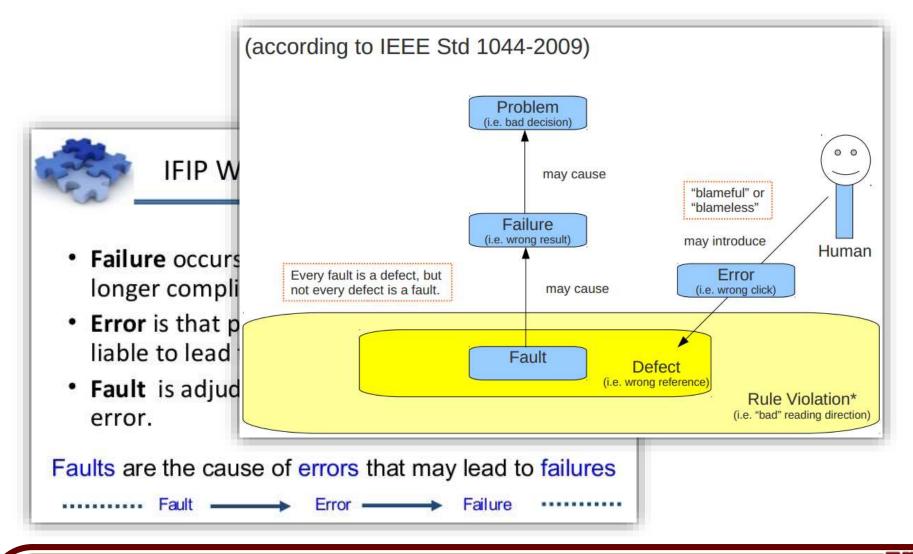
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Fault – Error – Failure – Problem

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Safety assessment

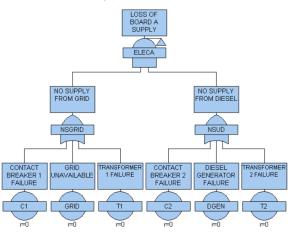
assessment

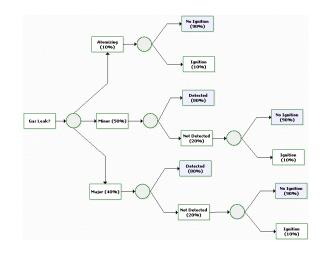
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evidence

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Definition and Comments	Relationships
The process, and the result, of	Supports
analysing systematically and evaluating the hazards associated with sources and practices, and associated protection and safety measures.	• Safety case
Assessment is often aimed at	Requires
quantifying performance	Risk assessment
measures for comparison with criteria.	Failure modes
 Deterministic safety 	Basic event probabilities
assessment	 Safety case
 Probabilistic safety 	 Safety arguments and





ШE

Safety case

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Definition and Comments	Relationships
A collection of arguments	Types of claims
and evidence in support of the safety of a facility or activity.	 Reliability-functionality Safety-robustness Safety-fail safe Rule compliance Vulnerability assessment
Property-based, vulnerability	Sources of evidence
aware, standards-informed and is described by the safety justification triangle.	 E.g. Functionality: Random testing Statistical testing Functional testing Model-based testing Development metrics Static analysis
	 Formal verification
	 Modelling and simulation



Verification and validation

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Definition and Comments	Relationships		
Validation The process of determining	Validation is broader in scope, than verification.	Examples	
whether a product or service is adequate to perform its intended function satisfactorily.	 Computer system validation: testing and evaluation of the integrated computer system to ensure compliance with the requirements. 	SimulationEmulationTesting	
Verification The process of determining whether the quality or	Verification is closely related to quality assurance and quality control.	Examples	
performance of a product or service is as stated, as intended or as required.	 Computer system verification: ensuring that a phase in the system life cycle meets the requirements imposed on it by the previous phase. 	 Specification analysis Static analysis Model-based development Formal verification 	

